

or Type A test, measures overall containment leakage. However, operating experience with all types of containments used in this country demonstrates that essentially all containment leakage can be detected by local leakage rate tests (Types B and C). According to results given in NUREG-1493, out of 180 ILRT reports covering 110 individual reactors and approximately 770 years of operating history, only 5 ILRT failures were found which local leakage rate testing could not detect. This is 3% of all failures. This study agrees well with previous NRC staff studies which show that Types B and C testing can detect a very large percentage of containment leaks. The Millstone Unit 3 experience has also been consistent with these results.

The Nuclear Management and Resources Council (NUMARC), now the Nuclear Energy Institute (NEI), collected and provided the NRC staff with summaries of data to assist in the Appendix J rulemaking effort. NUMARC collected results of 144 ILRTs from 33 units; 23 ILRTs exceeded 1.0L_a. Of these, only nine were not due to Type B or C leakage penalties. The NEI data also added another perspective. The NEI data show that in about one-third of the cases exceeding allowable leakage, the as-found leakage was less than 2L_a; in one case the leakage was found to be approximately 2L_a; in one case the as-found leakage was less than 3L_a; one case approached 10L_a; and in one case the leakage was found to be approximately 21L_a. For about half of the failed ILRTs the as-found leakage was not quantified. These data show that, for those ILRTs for which the leakage was quantified, the leakage values are small in comparison to the leakage value at which the risk to the public starts to increase over the value of risk corresponding to L_a (approximately 200L_a, as discussed in NUREG-1493).

The licensee also addressed the possible increase in risk due to extending this test interval. The licensee concluded that any increase in risk would be negligible. This is consistent with independent staff studies documented in NUREG-1493.

Therefore, based on these considerations, it is unlikely that an extension of one cycle for the performance of the Appendix J, Type A test at Millstone Unit 3 would result in significant degradation of the overall containment integrity. Likewise, performance of the third test in a refueling outage other than when the plant is shut down for the 10-year plant inservice inspections has no connection to the detection of overall containment

degradation. As a result, the application of the regulation in these particular circumstances is not necessary to achieve the underlying purpose of the rule.

The preoperational Type A test required by Appendix J was performed in July 1985. Millstone Unit 3 started commercial operation on April 23, 1986. The staff considers this date to also be the start of the licensee's first 10-year Type A test period. The extension of the Type A test interval for Millstone Unit 3 discussed in this document is referenced to this starting date. Based on generic and plant specific data, the NRC staff finds the basis for the licensee's proposed exemptions to be acceptable.

Pursuant to 10 CFR 51.32, the Commission has determined that granting this Exemption will have no significant impact on the quality of the human environment (60 FR 22415).

This Exemption is effective upon issuance and shall expire at the completion of the 1997 refueling outage.

Dated at Rockville, Maryland, this 8th day of May 1995.

For the Nuclear Regulatory Commission.

Steven A. Varga,

*Director, Division of Reactor Projects—I/II,
Office of Nuclear Reactor Regulation.*

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Advisory Committee on Reactor Safeguards Subcommittee Meeting on Planning and Procedures; Notice of Meeting

The ACRS Subcommittee on Planning and Procedures will hold a meeting on June 7, 1995, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance, with the exception of a portion that may be closed pursuant to 5 U.S.C. 552b(c) (2) and (6) to discuss organizational and personnel matters that relate solely to internal personnel rules and practices of ACRS, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.

The agenda for the subject meeting shall be as follows:

Wednesday, June 7, 1995—2 p.m. Until the Conclusion of Business

The Subcommittee will discuss proposed ACRS activities and related matters. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff person named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

Further information regarding topics to be discussed, the scheduling of sessions open to the public, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements, and the time allotted therefore can be obtained by contacting the cognizant ACRS staff person, Dr. John T. Larkins (telephone: 301/415-7360) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any changes in schedule, etc., that may have occurred.

Dated: May 9, 1995.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

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PENSION BENEFIT GUARANTY CORPORATION

Request for Approval of a Modification in an Approved Collection of Information; PBGC Form 10-SP, Optional Reportable Event Form for Small Plans

AGENCY: Pension Benefit Guaranty Corporation.

ACTION: Notice of Request for OMB approval.

SUMMARY: The Pension Benefit Guaranty Corporation has requested that the Office of Management and Budget approve, under the Paperwork Reduction Act, a modification in its currently approved collection of information for the reporting requirements under section 4043 of the Employee Retirement Income Security Act of 1974 (OMB control number 1212-0013; expires February 28, 1996). This modification would simplify compliance for small plans by providing the plan administrator and contributing